



LinkedIn: Alice Appleby  
Email: axa1291@student.bham.ac.uk

A. Appleby <sup>a,b</sup>, R. Khan <sup>a</sup>, M. Attallah <sup>b</sup>

<sup>a</sup> TWI Ltd., Granta Park, Great Abington, Cambridge, CB21 6AL, UK

<sup>b</sup> School of Metallurgy and Materials, University of Birmingham, B15 2TT, Birmingham, UK

## Introduction

The development of nuclear powder infrastructure, both fission and fusion, is essential for a sustainable future. In order for these reactors to peak efficiency, they need to operate in extreme conditions posing a serious need to evaluate structural materials in use. One of these conditions is high temperatures (>550°C), for this reason **Oxide Dispersion Strengthened (ODS) steels** have been developed where nano-sized clusters of yttrium oxide are present within the microstructure impeding the movement of dislocations leading to **Orowan strengthening**. ODS steels are in development for fission and fusion reactors due to this increase in tensile and creep strength<sup>1,2</sup>, as well as increased resistance to irradiation induced degradation.

ODS steels are limited to powder manufacturing to ensure a fine and homogeneous distribution of nano-oxides. The production of ODS steels involves atomisation, mechanical alloying followed by consolidation. The atomisation techniques and mechanical alloying has significant effects on the subsequent powder characteristic such as morphology, size distribution, flowability and powder density. **Hot isostatic pressing** is a desirable consolidation method to produce fully dense ODS steels because it is a near-net shape process and produces isotropic properties. The effect of powder quality and HIP parameters on the resultant properties must be fully understood.

## Aim

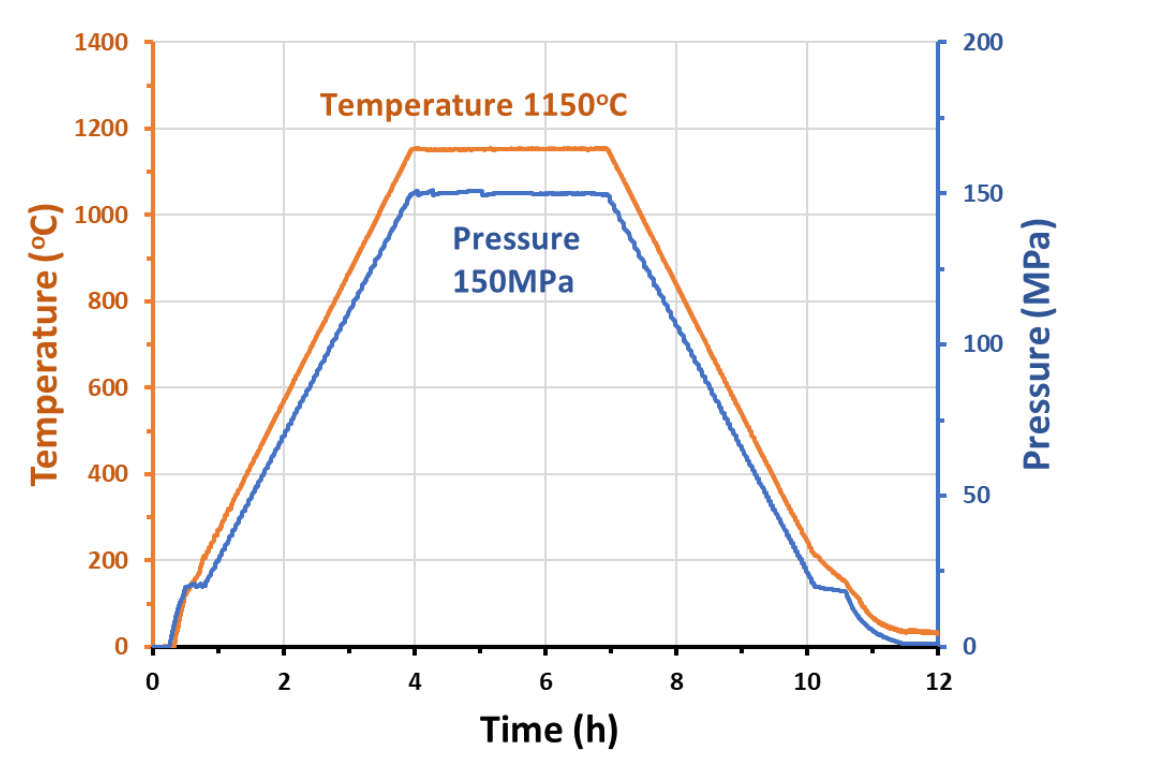
- Assess the effects of powder quality and hot isostatic pressing on the microstructure and structural integrity of steels utilised in the nuclear industry; PM2000 and Eurofer-97; in extreme environments such as high temperatures and exposure to proton irradiation.

## Materials

Both powders were gas atomised, the PM2000 powder was subsequently mechanically alloyed to introduce nano-sized yttrium oxide resulting in an ODS steel.

Alloy	Fe	Cr	Al	Ti	Y <sub>2</sub> O <sub>3</sub>	Other
Eurofer-97	Bal.	9.2	<0.01	<0.01	0	W, Mn, V, Ni, Ta
PM2000	Bal.	20	5.5	0.5	0.5	

## Methods

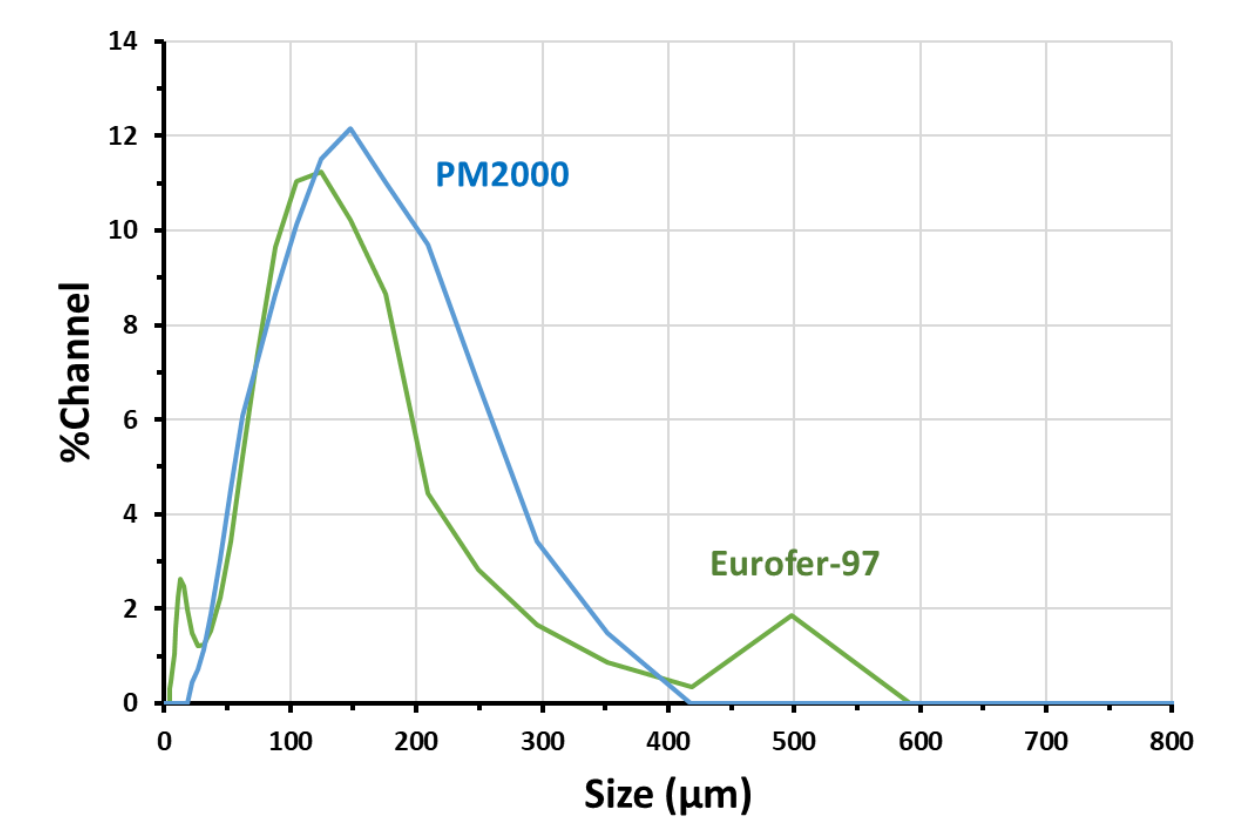


## Powder Characterisation

Property	PM2000	Eurofer-97
Flowability (s/50g)	29.7	34.0
Apparent density (g/cm <sup>3</sup> )	3.1	5.3
Tap density (g/cm <sup>3</sup> )	3.8	5.8

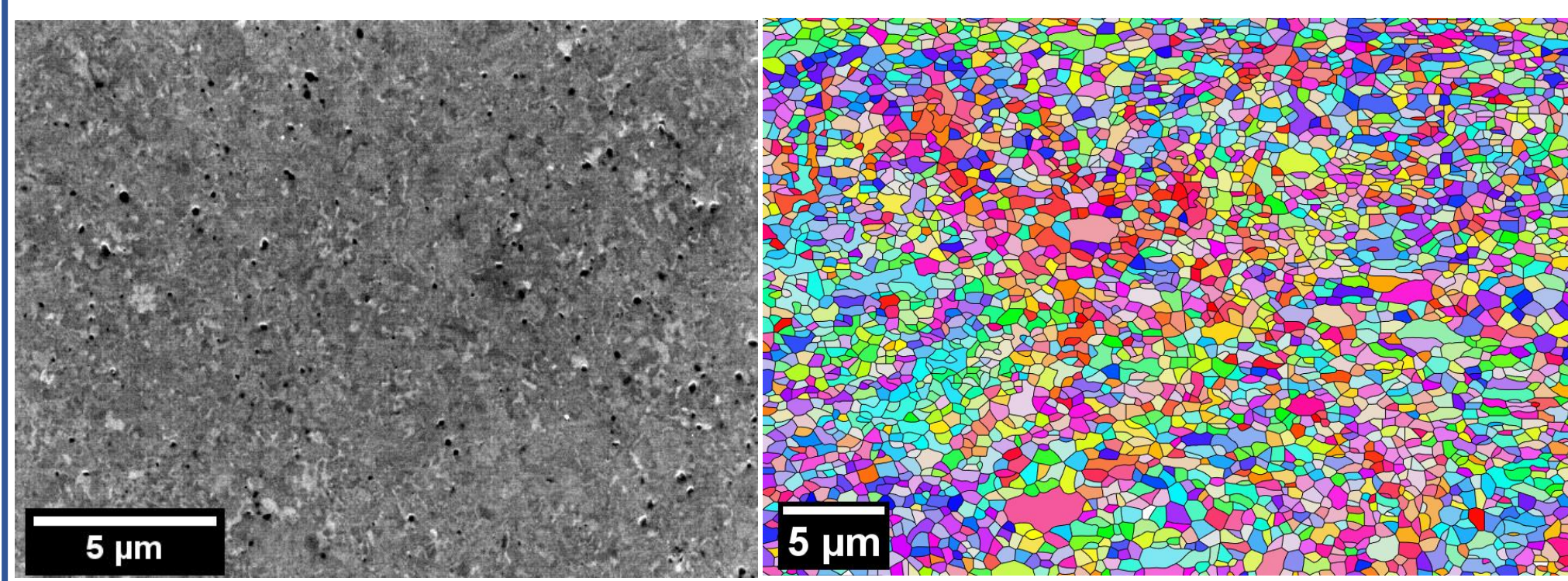
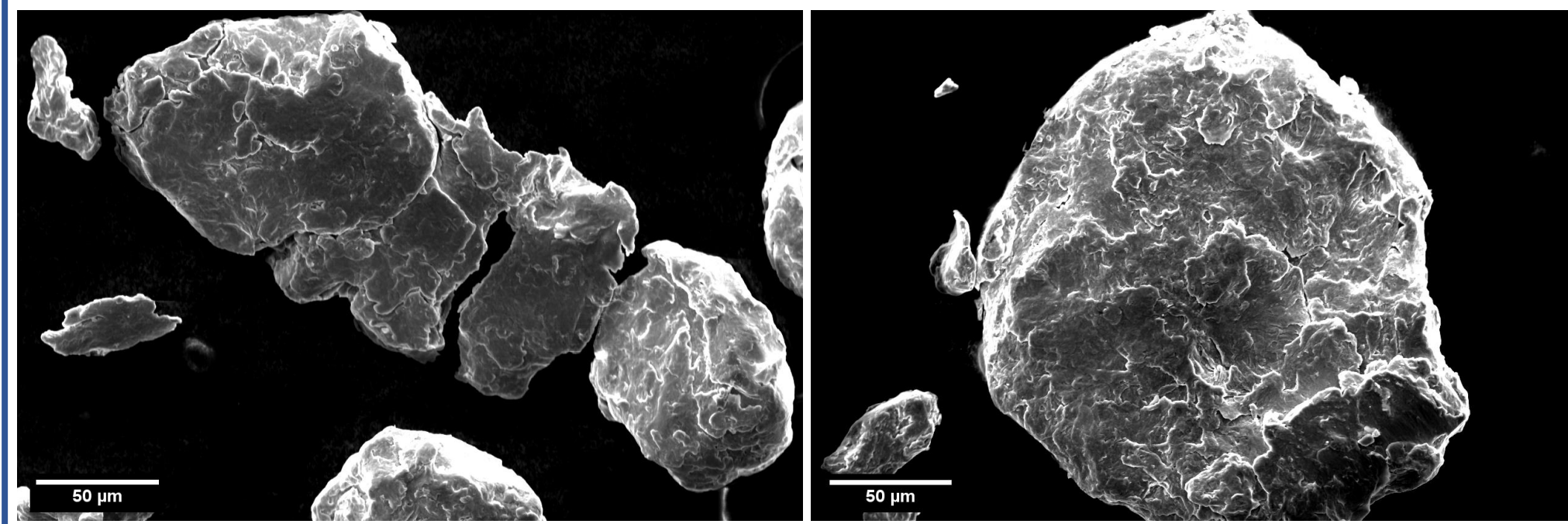
  

PSD Percentile	PM2000	Eurofer-97
D10	49	14
D50	115	92
D90	217	188

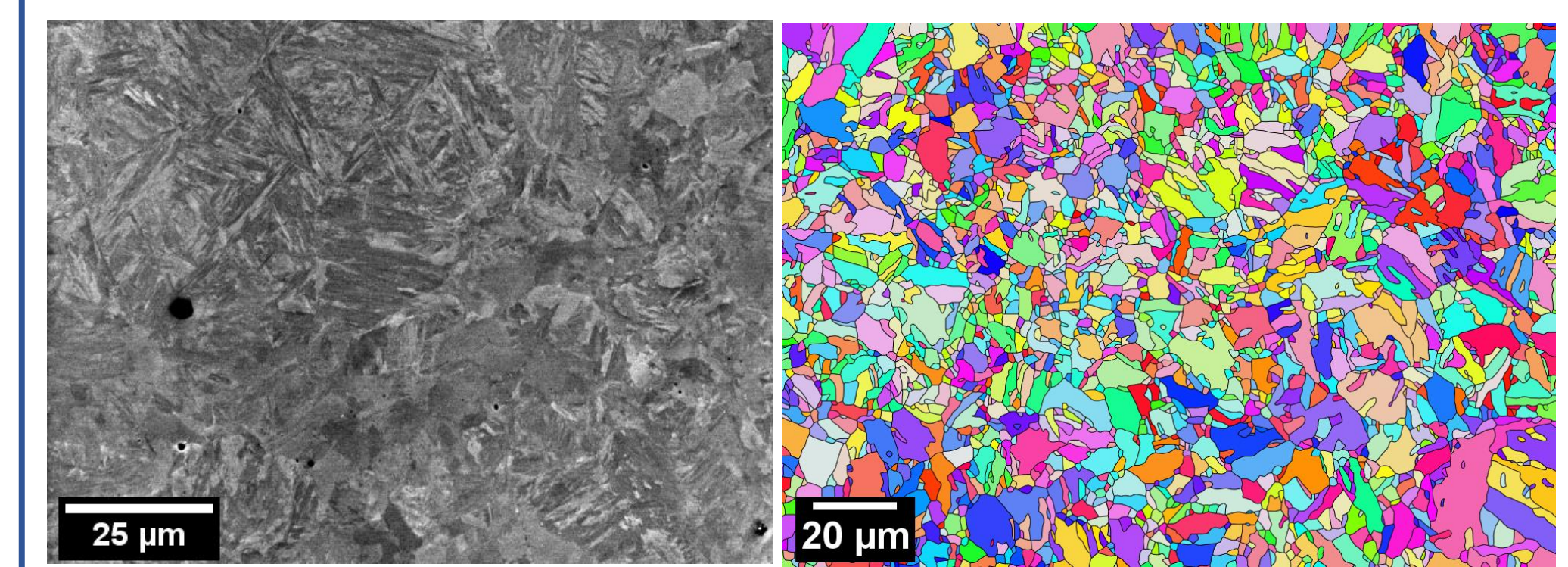
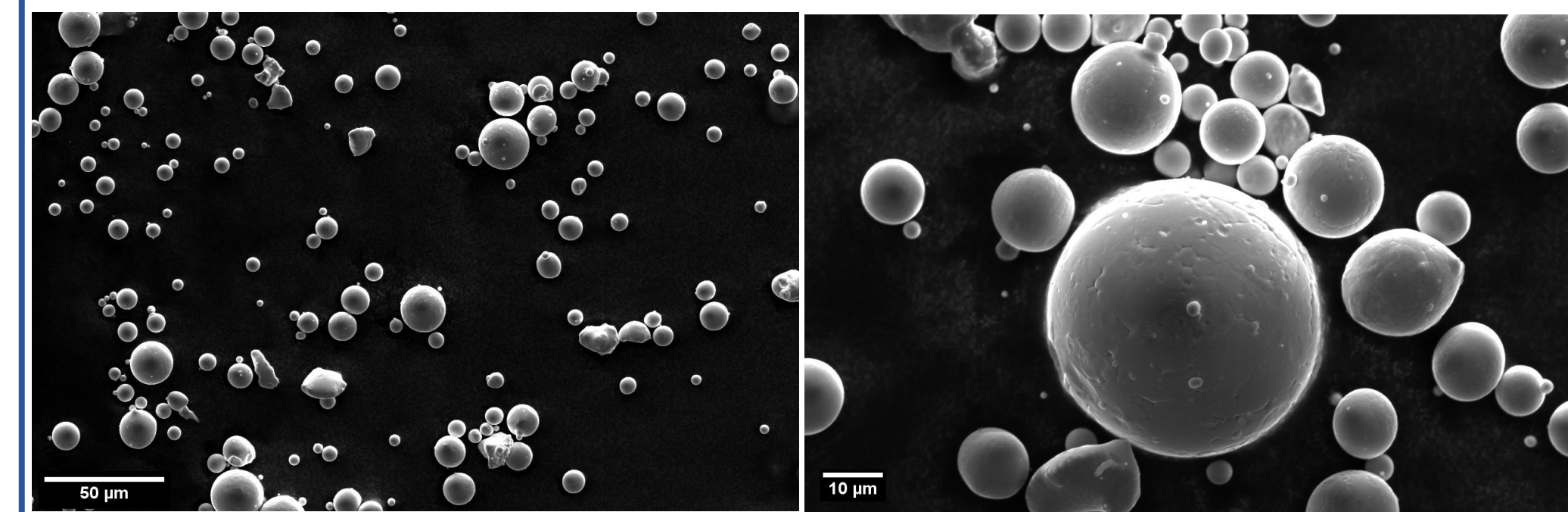


Particle size distribution of PM2000 and Eurofer-97

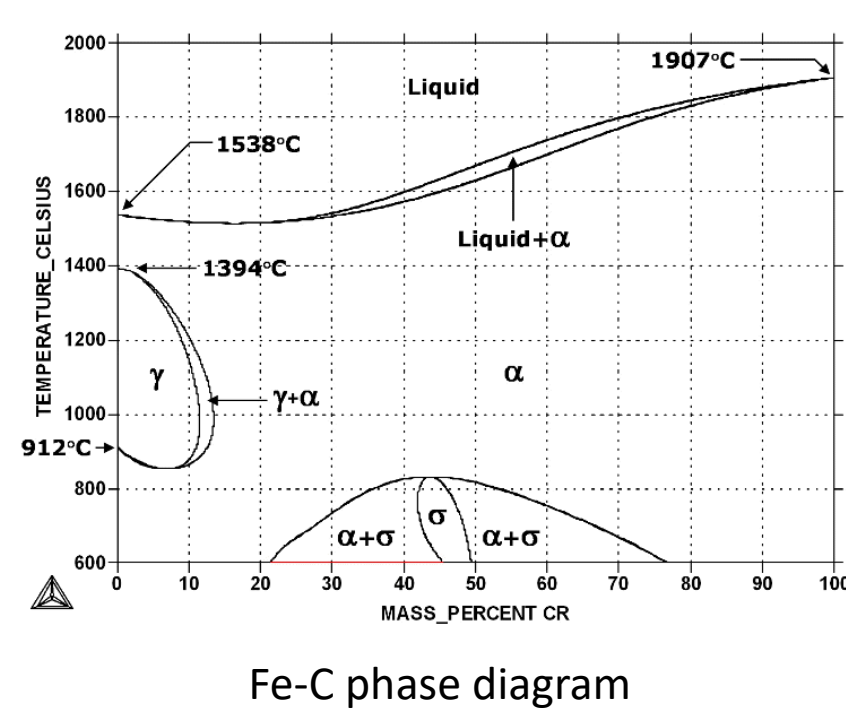
## PM2000



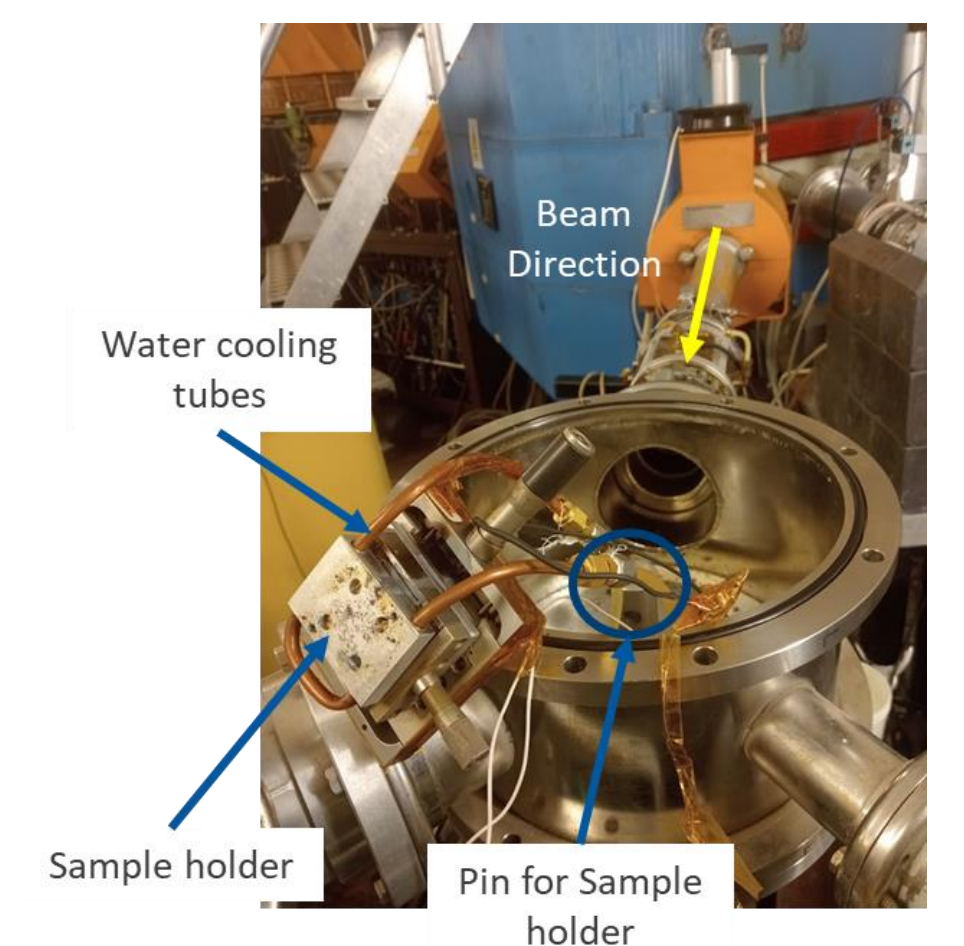
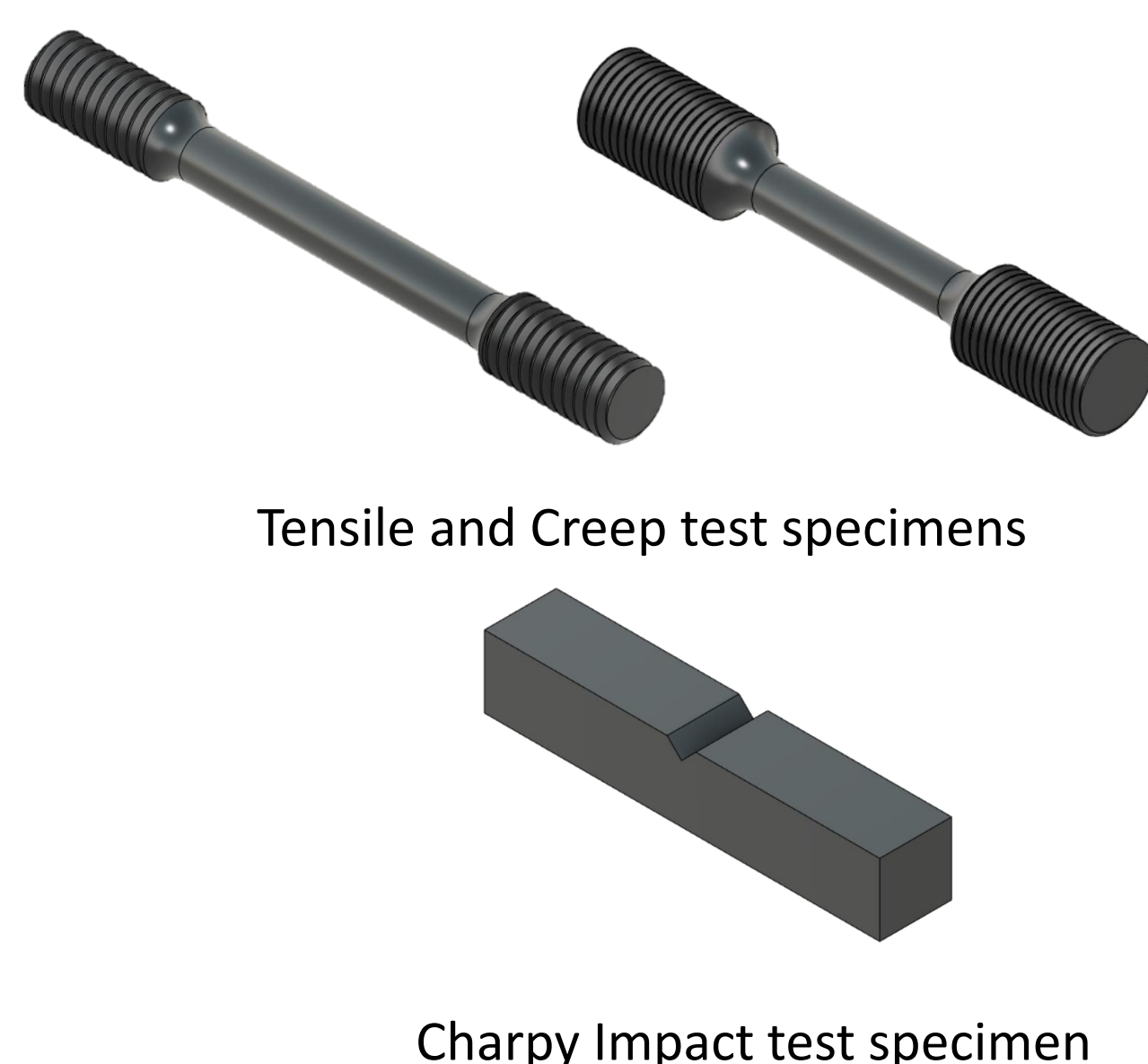
## Eurofer-97



## Future Work



Heat treatment for Eurofer-97: 950°C+WQ (austenising), 750°C/AC (tempering)



Irradiation parameters: 2.9MeV proton beam for 15 hours to reach a peak of 1dpa in both PM2000 and Eurofer-97 at approximately 30µm. The sample will be held at 300°C.

Production of ODS Eurofer-97

Mechanical Testing

Irradiation Testing



Lloyd's Register Foundation helps to protect life and property by supporting engineering-related education, public engagement and the application of research.



This publication was made possible by the support of TWI and Lloyd's Register Foundation. This work was enabled through, and undertaken at, the National Structural Integrity Research Centre (NSIRC), a postgraduate engineering facility for industry-led research into structural integrity established and managed by TWI through a network of both national and international Universities. This research is funded by TWI's Core Research Programme, a market-driven programme of research and development activities that underpin the creation and optimisation of joining, materials and engineering technologies. [www.twi-global.com/crp](http://www.twi-global.com/crp)

- Ukai, S., et al., *ALLOYING DESIGN OF OXIDE DISPERSION-STRENGTHENED FERRITIC STEEL FOR LONG-LIFE FBRS CORE MATERIALS*, Journal of Nuclear Materials, 1993, 204: p. 65-73.
- Kim, G. E., T. K. Kim, and S. Noh. 2020. 'DIFFERENCE OF MICROSTRUCTURES AND MECHANICAL PROPERTIES BETWEEN 9Cr-1W FERRITIC/MARTENSITIC STEEL AND ODS STEEL', *Archives of Metallurgy and Materials*, 65: 337-41.